



U.S. DEPARTMENT OF
ENERGY

Nuclear Energy

Fuel Cycle Research and Development

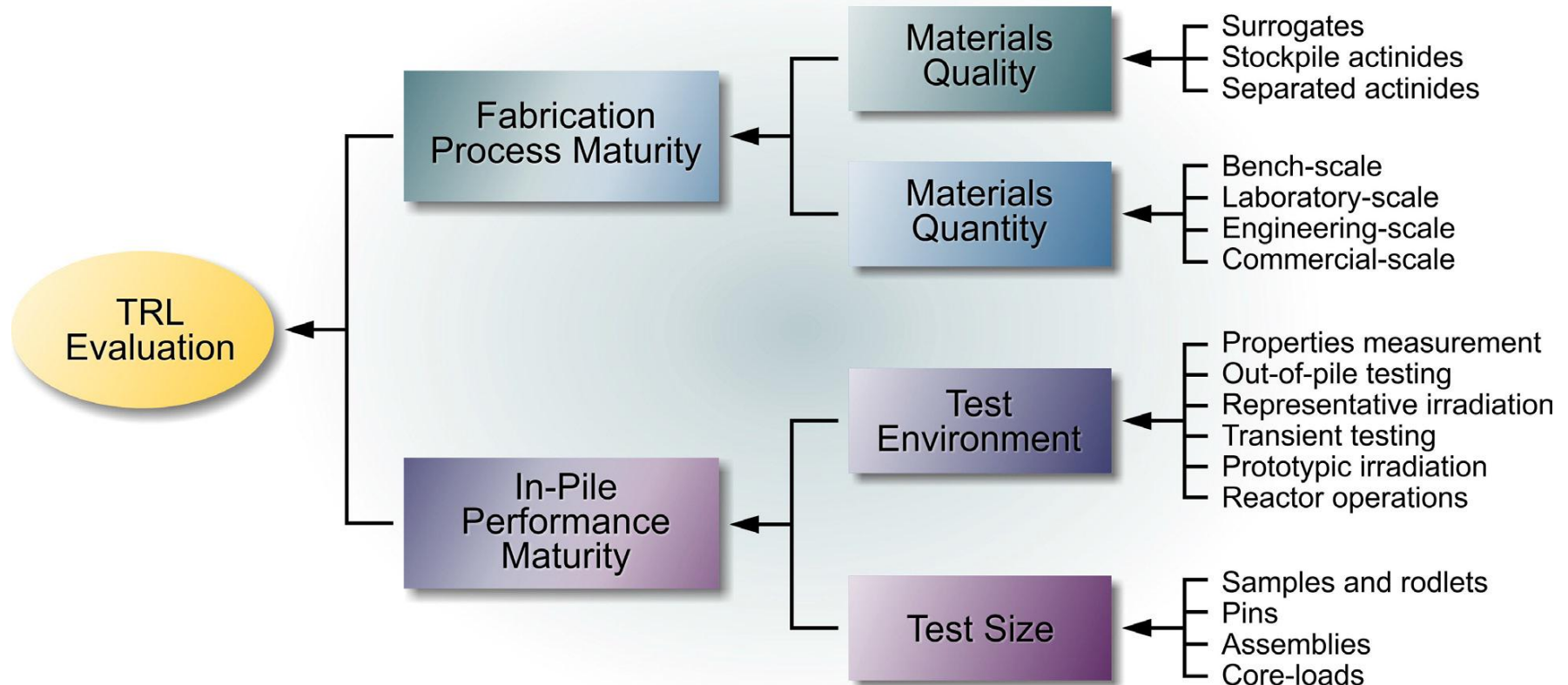
Technology Readiness Levels for Advanced Nuclear Fuel and Materials

Shannon M. Bragg-Sitton, Ph.D.
Deputy National Technical Director

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Idaho National Laboratory







Fabrication Process Maturity

- **Measures how well the fabrication process is understood and validated**

Attributes to be considered:

- **Quality of materials used for fabrication process development and testing**
 - Surrogate materials – non-radioactive or less-radioactive materials with properties similar to the actual actinides; selection of surrogate depends on phenomena of interest
 - Representative materials from stockpile – individually separated actinide elements that must be blended together for use in fuel fabrication experiments; generally requires a non-prototypic conditioning step; isotopic vectors typically different
 - Representative materials from recycling process – optimization of fabrication process using actinides recovered from separations processes, co-precipitated through a prototypic process and conditioned for use as feedstock
- **Quantity of materials for fabrication process development and testing**
 - Bench-scale – up to 100 g batch sizes (e.g. multiple pellets/year)
 - Lab-scale – up to 1 kg quantity batch sizes, ~10 kg TRU/yr (e.g. multiple pins/year)
 - Engineering-scale – 1-10 kg batch sizes, ~100 kg TRU/yr (e.g. a few assemblies/year)
 - Commercial-scale – 1-10 kg batch sizes, throughput rate ~tons TRU/yr (e.g. a few hundred assemblies/year – core loads)



- **Measures how well the in-pile performance of the fuel is understood and validated**

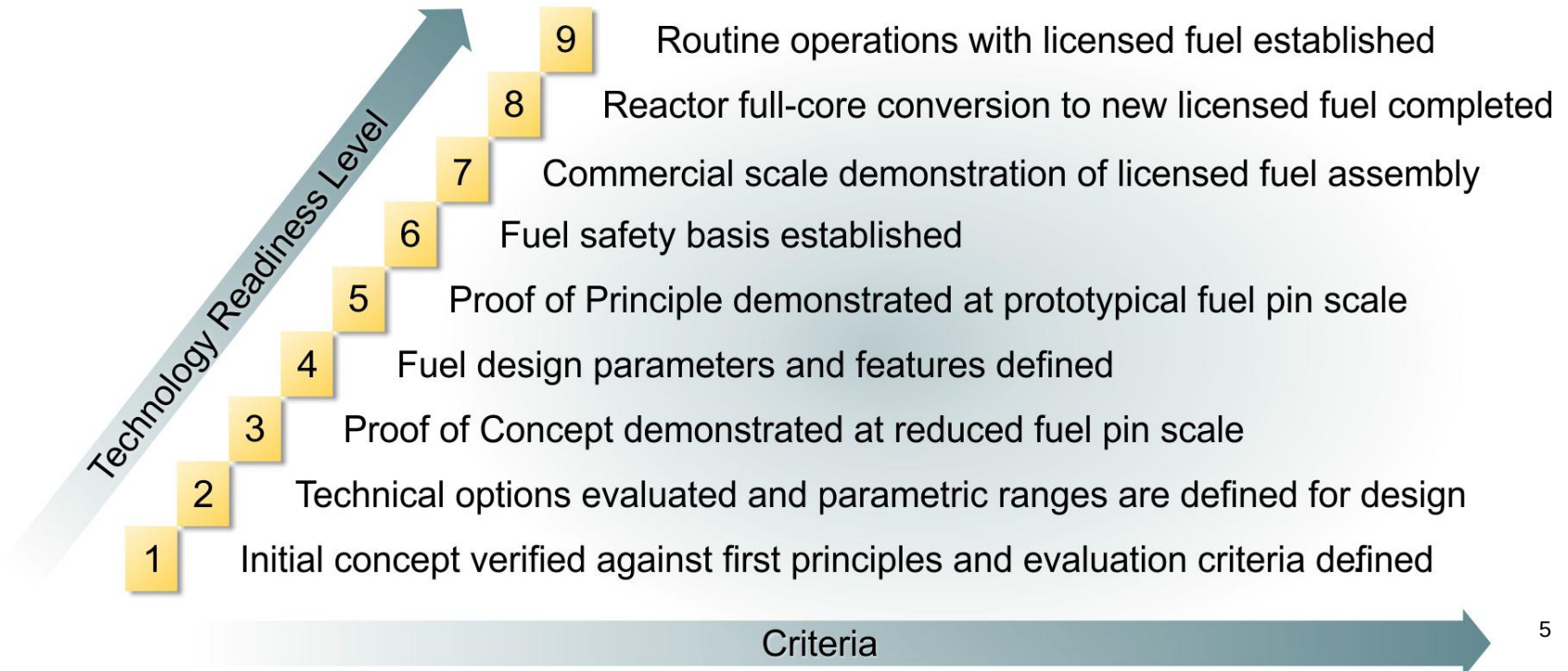
Parameters to be considered:

- **Test environment to determine performance parameters**
 - Fundamental property measurements
 - Out-of-pile testing
 - In-pile testing in representative spectrum
 - Transient testing
 - In-pile testing in prototypic spectrum
 - Reactor operations
- **Size of test campaigns to assess performance parameters**
 - Samples and rodlets
 - Pins – close to full-length, representative fuel heights and plenum volumes; prototypic cladding
 - Assemblies – prototypic design, lead fuel assemblies
 - Full-core assemblies – operations program; statistical and long-term reliability related issues quantified



■ High level TRL completion criteria:

- TRL evaluation based on binning the state-of-knowledge for each attribute
- Activities required to achieve goals identified as a function of technical elements, attributes and bins
- TRL Advancement requires development to the point of both fabrication and performance maturity





TRL Binning for Fabrication Process Maturity

Representative materials from separations process	TRL 4 - 5	TRL 6	TRL 7	TRL 8 - 9
Representative materials from stockpile	TRL 4	TRL 4 - 5		
Surrogate materials	TRL 3			
	Bench-scale (1 g - 1 kg)	Laboratory-scale (1 - 10 kg)	Engineering-scale (10 - 100 kg)	Commercial-scale (> 1 ton)
	Approximate Yearly Throughput			



TRL Binning for Irradiation Performance Maturity

Multiple assemblies (core loads)						TRL 8 - 9
Few assemblies					TRL 7	TRL 8
Pins		TRL 4	TRL 5	TRL 5 - 6	TRL 6	
Samples and rodlets	TRL 4	TRL 4	TRL 4	TRL 5	TRL 5	
	Fundamental property measurements	Out-of-pile testing	In-pile testing representative spectrum	Transient testing	In-pile testing prototypic spectrum	Reactor operations



Summary of criteria and activities for each TRL, specific to advanced fuels

Nuclear Energy

TRL Function

Definition

1	Proof-of-Concept	A new concept is proposed. Technical options for the concept are identified and relevant literature data reviewed. Criteria developed.
2		Technical options are ranked. Performance range and fabrication process parametric ranges defined based on analyses.
3		Concepts are verified through laboratory-scale experiments and characterization. Fabrication process verified using surrogates.
4	Proof-of-Principle	Fabrication of samples using stockpile materials at bench-scale. Irradiation testing of small-samples (rodlets) in relevant environment. Design parameters and features established. Basic properties compiled.
5		Fabrication of pins using prototypic feedstock materials at laboratory-scale. Pin-scale irradiation testing at relevant environment. Primary performance parameters with representative compositions under normal operating conditions quantified. Fuel behavior models developed for use in fuel performance code(s).
6		Fabrication of pins using prototypic feedstock materials at laboratory-scale and using prototypic fabrication processes. Pin-scale irradiation testing at relevant and prototypic environment (steady-state and transient testing). Predictive fuel performance code(s) and safety basis established.
7	Proof-of-Performance	Fabrication of test assemblies using prototypic feedstock materials at engineering-scale and using prototypic fabrication processes. Assembly-scale irradiation testing in prototypic environment. Predictive fuel performance code(s) validated. Safety basis established for full-core operations.
8		Fabrication of a few core-loads of fuel and operation of a prototype reactor with such fuel.
9		Routine commercial-scale operations. Multiple reactors operating.

← **LWR Accident Tolerant Fuels**

Transmutation Fuel

TRU-metal, TRU-oxide

(roughly same TRL)

Metal experience: mostly U.S.

Oxide experience: mostly International (France and Japan)

Fast Reactor Metallic U-Pu-Zr

- Not formally licensed for a full core load
- Not used in industrial-scale

Fast Reactor Metallic (U-Zr), Oxide (U, Pu)

- Licensed for reactor operations
- Successful mission operations
- Operational database wider for MOX, especially considering International experience

← **LWR UO₂-Zr Fuels**